



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**

REGION I
475 ALLENDALE ROAD
KING OF PRUSSIA, PA 19408-1415

February 11, 2004

Docket No. 05000309
License No. DPR-36
ISFSI Docket No. 07200030

Michael J. Meisner, Chief Nuclear Officer
Maine Yankee Atomic Power Company
321 Old Ferry Road
Wiscasset, ME 04578-4922

SUBJECT: INTEGRATED INSPECTION NO. 05000309/2003003

Dear Mr. Meisner:

On January 11, 2004, we completed an integrated inspection at your Maine Yankee reactor facility of activities authorized by the above listed NRC license. We discussed our findings with Mr. Ted Feigenbaum, Mr. Thomas Williamson, and others on February 10, 2004. The enclosed report presents the results of this inspection.

During this approximately four-month inspection period, we inspected your operations and maintenance, engineering, security and safeguards, emergency preparedness, and plant support programs through selective examinations of procedures and representative records, interviews with personnel, and observations by the inspectors. We consider the programs to be appropriately implemented.

In accordance with Section 2.790 of the NRC's "Rules of Practice," Part 2, Title 10, Code of Federal Regulations (CFR), a copy of this letter and its enclosure will be made available electronically for public inspection in the NRC Public Document Room or from the NRC's document system (ADAMS), accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html>.

We appreciate your cooperation with us during this inspection.

Sincerely,

/RA/

Ronald R. Bellamy, Chief
Decommissioning Branch
Division of Nuclear Materials Safety

Michael Meisner

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Enclosure:

Inspection Report No. 05000309/2003003

cc w/encl:

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Friends of the Coast

Michael Meisner

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U.S. NUCLEAR REGULATORY COMMISSION
REGION I

INSPECTION REPORT

Inspection No. 05000309/2003003

Docket Nos. 05000309 & 072-30

License No. DPR-36

Licensee: Maine Yankee Atomic Power Company

Location: 321 Old Ferry Road
Wiscasset, ME 04578-4922

Inspection Dates: September 15, 2003 - January 11, 2004

Inspectors: Mark C. Roberts, Senior Health Physicist
Decommissioning Branch
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Randolph C. Ragland, Jr., CHP, Health Physicist
Decommissioning Branch, DNMS

John Buckley, Maine Yankee Project Manager
Division of Waste Management
Office of Nuclear Materials Safety and Safeguards (NMSS)

Approved By: Ronald R. Bellamy, PhD, Chief
Decommissioning Branch, DNMS, Region I

EXECUTIVE SUMMARY

Maine Yankee Atomic Power Company
NRC Inspection Report No. 05000309/2003003

This integrated inspection included aspects of licensee operations & maintenance, engineering, security and safeguards, emergency preparedness, and plant support programs. The report covers approximately a four-month period of announced inspections by two regional inspectors, one NMSS inspector, and a four-person inspection team from the Oak Ridge Institute for Science and Education's (ORISE) Environmental Survey and Site Assessment Program (ESSAP).

Operation & Maintenance

The inspector reviewed selected security-related Condition Reports from the licensee's corrective action program and examined timeliness and effectiveness of the corrective actions implemented. No findings of significance were identified.

Plant Support

During the week of September 15, 2003, NRC staff from NMSS and Region I and the staff from NRC's contractor ORISE ESSAP performed an in-process evaluation of Maine Yankee's Final Status Survey (FSS) program. Specific details of ESSAP's evaluation are contained in ORISE Report, "Confirmatory Survey of the Containment Spray Building Survey Units 6, 7, and 8, and In-Process Inspection Evaluation of the Eberline E-600, Maine Yankee Atomic Power Company, Wiscasset, Maine, November 2003" (ADAMS Accession Number ML040350018). No findings of significance were identified. A summary of the inspection results are as follows:

Maine Yankee implemented an effective final status survey program for the survey and release of station buildings and grounds. FSS data packages were complete, survey instruments were appropriately maintained and calibrated, and records for FSS technician training and qualification were up-to-date and appropriately maintained.

NRC/ORISE radiological surveys performed in Maine Yankee Containment Spray Building Survey Units 6, 7, and 8 confirmed that radiological conditions met NRC approved site specific release criteria. A comparison of ORISE and Maine Yankee data also showed close agreement.

Maine Yankee demonstrated that they could reliably detect radioactivity equivalent to 5.9 picocuries per gram (pCi/g) of Cs-137 per gram of soil (the scan Minimum Detectable Concentration (MDC) for Cs-137) with an E-600/SSPA-3 detector combination using standard Maine Yankee procedures for land scan surveys.

Maine Yankee's sediment removal activities conducted in the forebay and seal pit areas effectively and safely removed the upper layers of sediment containing detectable concentrations of Co-60 and Cs-137. The sediments were processed to remove free-standing water and disposed as radioactive wastes. Gamma spectrometry of sediment samples collected by the State of Maine and analyzed by the State and the NRC contractor, ORISE, showed good agreement. No findings of significance were identified.

Maine Yankee had properly classified and labeled a rail shipment of concrete wastes that was subsequently involved in an incident in a rail yard in South Portland, ME. Maine Yankee was notified of the event and dispatched Radiation Protection (RP) staff to the site to perform radiological measurements. The measurements confirmed that radioactive contamination was not present on the exterior of or outside the railcars. No findings of significance were identified.

Maine Yankee collected, sampled, and released dust suppression water from interior containment demolition activities in accordance with applicable procedural and regulatory requirements. No findings of significance were identified.

Emergency Preparedness

Maine Yankee conducted a medical emergency response drill that exercised command and control, medical, radiation protection, and security responses to a simulated onsite injury. No findings of significance were identified.

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REPORT DETAILS

Summary of Facility Activities

The plant was maintained in a permanently shutdown condition during this inspection period. Maine Yankee continued the transfer of spent fuel from their spent fuel pool into dry cask storage at their Independent Spent Fuel Storage Installation (ISFSI). Remediation activities, including surface scabbling and debris removal, continued in the Containment Spray Building and Primary Auxiliary Building, and demolition activities continued in the interior of the Containment Building.

I. Operations & Maintenance

O1 Conduct of Operations & Maintenance

O1.1 Corrective Action Program Review

a. Scope (Inspection Procedure (IP) 40801)

The inspector selected records from the licensee's corrective action program (Condition Reports (CRs)) for review and follow-up. Information was gathered through a review of documents, interviews with cognizant personnel, and tours of the facility.

b. Observations and Findings

Maine Yankee has developed a corrective action program to identify and correct conditions that are deemed adverse to quality; including procedural, industrial safety, radiological safety, and equipment issues. Issues identified are assigned an initial significance level and responsible department for subsequent action. A review board confirms the significance level and follow-up actions. Responsible departments typically have 30 working days to complete corrective actions on a CR. Maine Yankee has typically processed approximately 400 CRs annually.

After reviewing the 2003 calendar year summary list of 329 CRs generated through the end of October, the inspector selected 22 CRs with Security designated as the responsible department for follow-up because the issues identified covered a variety of issues including training, safety, radiation protection, procedure compliance, staffing, and access control issues. Corrective actions for all 22 CRs were completed within the designated time-frame. Corrective actions for some of the issues were accomplished through a review of practices and procedures. Additional training was performed on an as-needed basis. Procedure changes or clarifications were discussed during the pre-shift security briefings, and if necessary, recorded in the routine post orders.

c. Conclusion

The inspector reviewed selected security-related CRs from the licensee's corrective action program and examined timeliness and effectiveness of the corrective actions implemented. No findings of significance were identified.

IV. Plant Support

R1 Radiation Protection & Chemistry (RP&C) Controls

R1.1 Final Status Survey Program Review & NRC Confirmatory Surveys

a. Scope (IP 83801)

During the week of September 15, 2003, NRC staff from the Office of Nuclear Material Safety & Safeguards and the NRC Region I Office and NRC's contractor, the Oak Ridge Institute for Science and Education's (ORISE's) Environmental Survey and Site Assessment Program (ESSAP), performed a review of Maine Yankee's implementation of their Final Status Survey (FSS) program. NRC staff performed a quality assurance review of FSS records and ORISE staff performed independent radiological surveys of Containment Spray Building Survey Units 6, 7, and 8. ORISE staff also performed an in-process evaluation of Maine Yankee's use of the Thermo Electron Corporation E-600 digital survey meter for the performance of final status surveys. Information was gathered through reviews of documents, interviews with cognizant personnel, direct observations of work, and performance of in-process and confirmatory radiological surveys.

b. Observations and Findings

Containment Spray Building

Maine Yankee's Containment Spray Building (CSB) formerly contained low pressure safety injection pumps, containment spray pumps, and heat exchangers. The lower elevations of the CSB historically contained elevated levels of loose radioactive contamination (i.e., often greater than 500,000 disintegrations per 100 square centimeters (dpm/100 cm²). The lower elevation of the CSB was the first structure designated as a Class I Area (contamination above release criteria) in accordance with the guidance in NUREG-1575, Rev. 1, "Multi-Agency Radiation Survey and Site Investigation Manual (MARSSIM)" to be remediated and released under the FSS program. Accordingly, NRC and ORISE reviewed Maine Yankee remediation and final status survey data for the CSB and also selected CSB Survey Units 6, 7, and 8 for independent confirmatory surveys.

NRC staff reviewed FSS data packages, instrument calibration records, and technician training and qualification records. The inspectors determined that all FSS survey instruments used by Maine Yankee were appropriately maintained and calibrated, and records for FSS technician training and qualification were up to date and appropriately maintained. FSS packages for CSB Survey Units 6, 7, and 8 were complete and had been reviewed and approved by Maine Yankee management. The inspectors did note that multiple minor corrections had been made to various forms in the FSS data packages by the FSS Manager after the data forms had been reviewed and approved by an independent reviewer. The FSS Manager acknowledged opportunities for improvement in the FSS package review process.

At the start of the confirmatory survey on September 15, 2003, the NRC - ORISE inspection team found that all plant equipment had been removed from the CSB,

concrete surfaces were free of paint and other coatings, and the majority of concrete surfaces had been scabbled during remediation activities. Dust and debris from remediation activities had been removed, and remediation and final status surveys, and FSS data packages for CSB Survey Units 6, 7, and 8 were complete.

ORISE staff performed surface scans, direct measurements, gamma scans, and removable surface activity evaluations in CSB Survey Units 6, 7, and 8. Surfaces were scanned for total beta activity using gas-flow proportional detectors and scanned for gamma radiation using scintillation detectors. Total beta and gamma radiation scans were performed on approximately 90% of concrete floors and lower walls up to the 17-foot elevation. Particular attention was given to cracks and joints in concrete surfaces, scabbled surface areas, and other locations where radioactive material may have accumulated. All detectors were coupled to ratemeter-scalers with audible indicators. Smears were performed on surfaces to evaluate the presence of removable contamination. All samples and data were returned to ORISE's ESSAP laboratory in Oak Ridge, TN for analysis and interpretation. Samples were analyzed for gross alpha and gross beta activity using a low background gas-flow proportional counter. Smear data and direct measurements were converted to units of dpm/100 cm².

Maine Yankee's site specific derived concentration guideline levels (DCGL_W) for interior surfaces of the CSB, as specified in Table 6-11 of Maine Yankee's License Termination Plan (LTP) is 18,000 dpm/100 cm² (gross beta activity) above background. The derived concentration guideline level for elevated measurements comparison (DCGL_{EMC}) was calculated by Maine Yankee to be 90,000 dpm/100 cm² for CSB Survey Units 6, 7, and 8.

ORISE beta surface scans of the floors and walls of Survey Units 6, 7, & 8 identified some areas of elevated radiation. The majority of the areas were identified in Survey Unit 8. All elevated areas were small in size, generally less than 300 cm². Gamma scans did not identify any indications of volumetric or subsurface contamination (i.e., gamma radiation levels were consistently within background ranges). Five measurements exceeded the DCGL_W and no measurements exceeded the DCGL_{EMC} of 90,000 dpm/100 cm². A comparison of ORISE and Maine Yankee data also showed close agreement for measurements performed in the same area. No significant removable beta-gamma or alpha surface activity was identified.

E-600 Evaluation

Maine Yankee utilizes Thermo Electron Corporation's E-600 digital survey meter coupled to scintillation detectors for the performance of land scan surveys. Maine Yankee's survey protocol relies on the electronic circuitry of the E-600 digital survey meter to identify areas of elevated activity rather than relying on the surveyor's recognition of an increase in an audible count rate to identify areas of elevated activity. During NRC Inspection No. 05000309/2003002 (ADAMS Accession Number ML032890079), Maine Yankee successfully demonstrated that they could reliably detect simulated buried hot particles using the E-600/SSPA-3 survey instrument using standard management-approved procedures for land scan surveys. During this inspection, Maine Yankee's objective was to demonstrate that they could reliably detect distributed radioactivity at a concentration equivalent to 5.9 picocuries per gram (pCi/g) of cesium-137 (Cs-137) using the E-600/SSPA-3 detector while following standard procedures for

land scan surveys. The value of 5.9 pCi/g is Maine Yankee's calculated and experimentally determined scan Minimum Detectable Concentration (MDC) for Cs-137.

To perform the demonstration, Maine Yankee constructed a 1-meter x 2-meter test pad containing soil and a measured amount of potassium chloride. Potassium chloride contains the natural radioactive isotope potassium-40, which served as the radioactive test source. Maine Yankee conducted modeling and determined that soil uniformly contaminated with 5.9 pCi/g of Cs-137 would result in a net instrument count rate above background of 2,785 net counts per minute (cpm) using the E-600/SSPA-3 detector. Maine Yankee then uniformly mixed potassium chloride into the test pad soil to achieve a E-600/SSPA-3 instrument count rate of less than or equal to 2,785 cpm. NRC and ORISE staff directly observed a Maine Yankee technician obtaining 15 static measurements across the 1-m x 2-m test pad. The collected data showed that count rates were uniform across the test pad and all count rates were less than 2,785 cpm.

NRC and ORISE staff then observed a Maine Yankee technician establish the E-600/SSPA-3 instrument background and alarm set-point by obtaining five one-minute background measurements in an unaffected area, and then setting the E-600/SSPA-3 alarm setpoint, according to FSS procedural guidance, equivalent to the average background count rate plus 3 standard deviations of the average background measurement.

NRC and ORISE staff then observed a Maine Yankee technician perform 25 scans of the test pad using the E-600/SSPA-3 detector combination. In each case, the E-600 meter alarmed early in the trial (i.e., within 10 seconds of commencing the scan). Each alarm was verified and confirmed with a second stage scan resulting in a second alarm in all 25 trials.

Based on a review of records including modeling calculations, instrument manuals, and procedures; and direct observations made during the demonstration, NRC and ORISE staff determined that Maine Yankee's demonstration was adequate to demonstrate the performance of the E-600/SSPA-3 detector combination for identifying distributed sources of radioactivity during the performance of land scan surveys. **Inspection Follow-up Item (IFI) 2000-03-02 is closed.**

c. Conclusions:

During the week of September 15, 2003, NRC staff from NMSS and Region I and the staff from NRC's contractor ORISE ESSAP performed an in-process evaluation of Maine Yankee's FSS program. Specific details of ESSAP's evaluation are contained in ORISE Report, "Confirmatory Survey of the Containment Spray Building Survey Units 6, 7, and 8, and In-Process Inspection Evaluation of the Eberline E-600, Maine Yankee Atomic Power Company, Wiscasset, Maine, November 2003" (ADAMS Accession Number ML040350018). No findings of significance were identified. A summary of the inspection results are as follows:

Maine Yankee implemented an effective final status survey program for the survey and release of station buildings and grounds. FSS data packages were complete, survey instruments were appropriately maintained and calibrated, and

records for FSS technician training and qualification were up-to-date and appropriately maintained.

NRC/ORISE radiological surveys performed in Maine Yankee Containment Spray Building Survey Units 6, 7, and 8 confirmed that radiological conditions met NRC approved site specific release criteria. A comparison of ORISE and Maine Yankee data also showed close agreement.

Maine Yankee demonstrated that they could reliably detect radioactivity equivalent to 5.9 pCi/g of Cs-137 per gram of soil (the scan MDC for Cs-137) with an E-600/SSPA-3 detector combination using standard Maine Yankee procedures for land scan surveys.

R1.2 Forebay and Seal Pit Remediation and Data Comparisons with State of Maine

a. Scope (IP 83801)

The inspector reviewed initial sediment removal, survey, and sampling activities conducted in the forebay and seal pit areas. Information was gathered through a review of documents and radiological sample data, tours of the remediation areas, reviews of selected underwater video tapes of the remediation activities, and discussions with cognizant personnel. In addition, seven sediment samples from the seal pit were collected by the State of Maine and split with the NRC. The State of Maine and NRC's contractor ORISE analyzed the samples by gamma spectrometry.

b. Observations and Findings

Routine liquid effluents were released through the forebay and seal pit areas from the Maine Yankee facility. Initial characterization of the area identified detectable concentrations of cobalt-60 (Co-60) and Cs-137 in upper sediment layers in these areas. Maine Yankee initiated a campaign to collect sediment by vacuum dredging. Sediments were then processed to remove free-standing water and shipped as radioactive waste.

After initial remediation activities were conducted in the seal pit area, the State of Maine conducted survey measurements in the area and collected sediment samples for gamma spectrometry analysis. Some additional remediation activities and final status surveys were performed by Maine Yankee following the State of Maine surveys. Selected samples were split with the NRC and submitted to the NRC's contractor, ORISE, for gamma spectrometry analysis. The data for Co-60 and Cs-137 concentrations for these samples are reported in the table below. The data show good agreement between the State of Maine and ORISE. It should be noted that although the samples were collected before additional remediation was completed and represent pre-FSS data, all of the results are below the $DCGL_{EMC}$ for the seal pit area.

COMPARISON OF STATE OF MAINE AND NRC CONTRACTOR (ORISE) GAMMA SPECTROMETRY DATA

Sample Identification	Radionuclide Concentrations (pCi/g) in Seal Pit Sediment ⁽¹⁾			
	NRC Contractor (ORISE) Data		State of Maine Data	
	Co-60	Cs-137	Co-60	Cs-137
S093	8.32 ± 0.36	2.93 ± 0.18	6.94 ± 0.31	2.59 ± 0.19
S202	4.14 ± 0.22	0.91 ± 0.10	4.87 ± 0.22	1.10 ± 0.08
S143	0.45 ± 0.06	0.77 ± 0.06	0.44 ± 0.02	0.68 ± 0.05
S160	3.62 ± 0.18	1.03 ± 0.09	3.66 ± 0.17	1.12 ± 0.08
S185	5.55 ± 0.26	1.05 ± 0.10	5.96 ± 0.27	1.00 ± 0.08
S188	2.89 ± 0.14	0.85 ± 0.07	2.77 ± 0.13	0.75 ± 0.06
S186	2.51 ± 0.16	0.48 ± 0.07	2.59 ± 0.12	0.60 ± 0.05

(1) Uncertainties are reported at the 95% confidence level,

c. Conclusion

Maine Yankee's sediment removal activities conducted in the forebay and seal pit areas effectively and safely removed the upper layers of sediment containing detectable concentrations of Co-60 and Cs-137. The sediments were processed to remove free-standing water and disposed as radioactive wastes. Gamma spectrometry of sediment samples collected by the State of Maine and analyzed by the State and the NRC contractor, ORISE, showed good agreement. No findings of significance were identified.

R1.3 Response to Railcar Transportation Incident

a. Scope (IP 86750)

The inspector reviewed the Maine Yankee response to an incident that occurred in the Rigby rail yard, in South Portland, ME, that involved two railcars loaded with concrete waste from the Maine Yankee decommissioning project. Information was gathered through a review of Condition Report 03-349, radiological survey records and photographs from the incident site, applicable regulations, and discussions with cognizant personnel.

b. Observations and Findings

The Maine Yankee Control Room was notified on the evening of November 17, 2003, that two gondola rail cars from a shipment of seven had been involved in an incident in the Rigby rail yard in South Portland. The shipment was parked on a siding in the rail yard. The cover of one gondola car was knocked off and a second cover on an adjacent car was damaged by a passing rail car containing scrap metal. Maine Yankee dispatched Radiation Protection (RP) staff to the site of the incident and confirmed that no visible concrete debris was outside the Maine Yankee gondola car. The Control Room also notified the Program Manager for the Maine Radiation Control Program of the event and a state inspector was dispatched to the rail yard. Radiological surveys performed by the RP staff confirmed that radioactive contamination levels were not in excess of background levels on any of the railcars, the dislodged cover, or on the ground adjacent to the railcars and railcar cover. The Maine Yankee RP staff placed a temporary cover on the open railcar and the two rail cars were subsequently returned to the Maine Yankee site the next day.

The inspector reviewed the classification and labeling for the radioactive waste shipment and found that the shipping documents accurately characterized the shipment in accordance with NRC requirements. Because the two damaged railcars contained wastes with radioactive material concentrations below 0.002 microcuries/gram, the Department of Transportation (DOT) regulations did not require classification of the wastes as hazardous material.

c. Conclusion

Maine Yankee had properly classified and labeled a rail shipment of concrete wastes that was subsequently involved in an incident in a rail yard in South Portland, ME. Maine Yankee was notified of the event and dispatched RP staff to the site to perform radiological measurements. The measurements confirmed that radioactive contamination was not present on the exterior of or outside the railcars. No findings of significance were identified.

R1.4 Liquid Effluent Release

a. Scope (IP 84750)

The inspector reviewed the collection, sampling, and release of dust suppression water collected from the sump in the reactor building. Information was gathered through a review of documents and radiological sample data, tours of the facility, and discussions with cognizant personnel.

b. Observations and Findings

Approximately 100,000 gallons of water used for dust suppression during interior containment demolition work was pumped from a sump in the containment building to a series of 20,000-gallon tanks. Maine Yankee received approval from the State of Maine to release this water through a modification of its effluent release permit. The inspector reviewed sample analysis data, release calculations, and observed selected procedural

steps during the release of water from one of the tanks. Radiation monitor readings on the effluent line indicated the effluent release was within regulatory limits.

c. Conclusion

Maine Yankee collected, sampled, and released dust suppression water from interior containment demolition activities in accordance with applicable procedural and regulatory requirements. No findings of significance were identified.

P1 Conduct of Emergency Preparedness

P1.1 Medical Emergency Response Drill

a. Scope (IP 88050)

The inspector reviewed the onsite response to an emergency preparedness medical drill involving a worker with simulated injuries and radioactive contamination. Information was gathered through a review of the response procedure and event scenario, direct observation of the drill and post-drill critique, and through interviews with cognizant personnel.

b. Observations and Findings

Maine Yankee conducted a drill involving a worker with simulated injuries and contamination to exercise their Medical Emergency Response procedure (2-50-8). Upon notification of the event, the Control Room dispatched a medical response team and a RP technician. The initial response to the injured worker appropriately considered both the worker's medical condition and the radiological conditions. Security staff expedited the arrival and departure of the ambulance dispatched to the site while maintaining appropriate security protocols for site entry. The post-drill critique provided an opportunity to review the response to the event.

c. Conclusion

Maine Yankee conducted a medical emergency response drill that exercised command and control, medical, radiation protection, and security responses to a simulated onsite injury. No findings of significance were identified.

V. Management Meetings

X1 Exit Meeting

The inspectors presented inspection results to representatives of the licensee's staff at the end of each inspection visit during the inspection period. On February 10, 2004, a summary of the inspection findings for the entire inspection period was presented to Mr. Ted Feigenbaum, Mr. Thomas Williamson, and others. Licensee representatives acknowledged the inspection findings.

PARTIAL LIST OF PERSONS CONTACTED

Licensee and Contractor Staff

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W. Ball, Director, Operations - ISFSI
R. Benner, Director, Decommissioning
L. Brown, Manager Safety
J. Connell, Radiation Protection Manager
K. Dinger, Independent Oversight Group
S. Evans, Environmental Remediation Manager
T. Feigenbaum, President
J. Grant, Security Director
J. Hebert, Regulatory Affairs
W. Henries, Director, Engineering
E. Howes, Director Public and Government Affairs
L. Jewett, Assistant Operations Manager
M. Meisner, Chief Nuclear Officer
E. Mercer, Radiological Engineering Supervisor
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J. Packer, Final Site Survey
T. Peacock, Manager, Site Restoration
G. Pillsbury, Engineer - Final Site Survey
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T. Shippee, QPD Manager
J. Temple, Emergency Preparedness
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T. Williamson, Director NSRA
P. Woodhams, Corrective Action Coordinator

State of Maine

P. Craighead, Maine Nuclear Safety Advisor
P. Dostie, Maine Nuclear Safety Inspector
C. Pray, Maine Nuclear Safety Advisor

INSPECTION PROCEDURES USED

IP 36801: Organization, Management & Cost Controls
IP 37801: Safety Reviews, Design Changes, and Mods at PSRs
IP 40801: Self-Assessment, Auditing, and Corrective Actions
IP 60855: Operation of an ISFSI
IP 83750: Occupational Radiation Exposure
IP 83801: Inspection of Final Status Surveys
IP 84750: Radwaste Treatment and Effluent & Environmental Monitoring
IP 86750: Solid Radwaste Management & Transportation of Radioactive Material
IP 88050: Emergency Preparedness

ITEMS OPENED, CLOSED, AND DISCUSSED

Items Opened: None

Items Closed: IFI 2000-03-01 Evaluation of Scan MDC Using the Eberline
E-600/SSPA-3

Items Discussed: None

LIST OF ACRONYMS USED

CFR	Code of Federal Regulations
cpm	counts per minute
CR	Condition Report
CSB	Containment Spray Building
DCGL _W	Derived Concentration Guideline Level (average for wide areas)
DCGL _{EMC}	Derived Concentration Guideline Level Elevated Measurement Comparison
DNMS	Division of Nuclear Materials Safety
DOT	Department of Transportation
dpm/100 cm ²	disintegrations per minute per 100 square centimeters
ESSAP	Environmental Survey & Site Assessment Program
FSS	Final Status Survey
IP	Inspection Procedure
ISFSI	Independent Spent Fuel Storage Installation
LTP	License Termination Plan
MARSSIM	Multi-Agency Radiation Survey & Site Investigation Manual
MDC	Minimum Detectable Concentration
MYAPS	Maine Yankee Atomic Power Station
NMSS	Office of Nuclear Materials Safety and Safeguards
NRC	Nuclear Regulatory Commission
ORISE	Oak Ridge Institute for Science and Education
PAB	Primary Auxiliary Building
PDR	Public Document Room
pCi/g	picocuries per gram
RP	Radiation Protection
RPC	Radiation Protection and Chemistry